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Advanced Techniques for Monte Carlo simulations: Ex-core responses supporting safety analysis

Monte Carlo Simulations supporting Safety Analyses

Complex nuclear systems:
Modelling irradiation field,
related responses



Modelling Multiplicative Systems
to get “far” targets
(ex-core detectors, fuel storage sites...)
Relevant for safety-security issues

Monte Carlo: Simulation Strategies

Determine system parameters
sampling “according to nature”

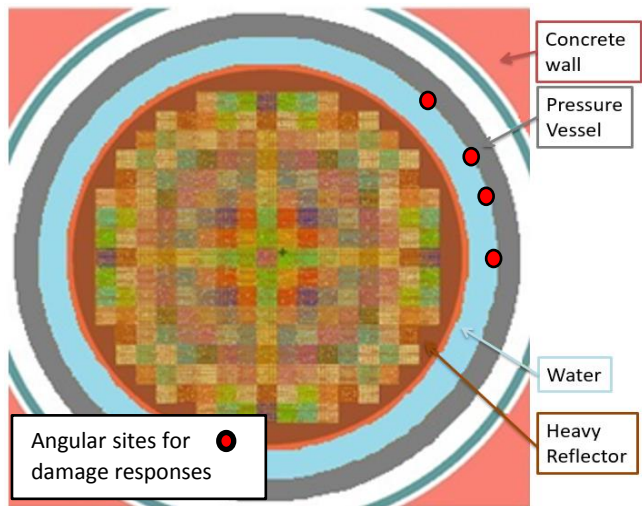
Determine detector response
biasing / forcing natural sampling

Standard Approach to Multiplicative Systems

Analog simulation gets neutron field
Source is fission reaction rate

Non-analog simulation gets “distant” responses
Variance Reduction Techniques

Standard vs. Proposed Approach: Decoupled vs. Single



Model of the core: MCNP(5-1.4 & 6.1)

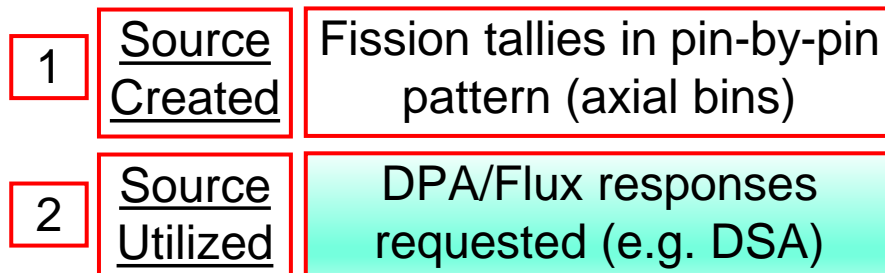
Reactor system: Gen-III/III+ core
Objective: dpa damage to vessel

Responses of interest:

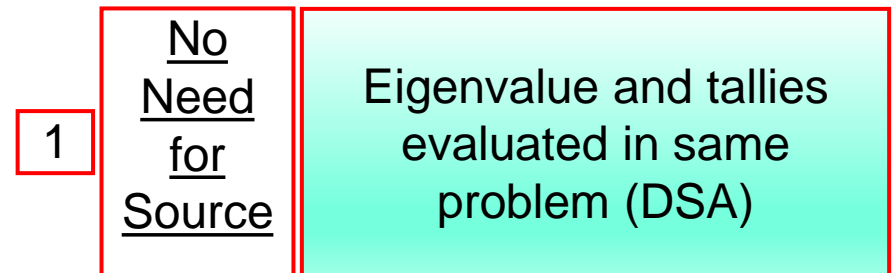
- Neutron flux ($E > 1 \text{ MeV}$)
- Neutron flux ($E > 100 \text{ keV}$)
- (Neutron dpa response function)
- Gamma flux ($E > 700 \text{ keV}$)

Evaluated at: core mid-plane, 4 angular positions

Standard Approach



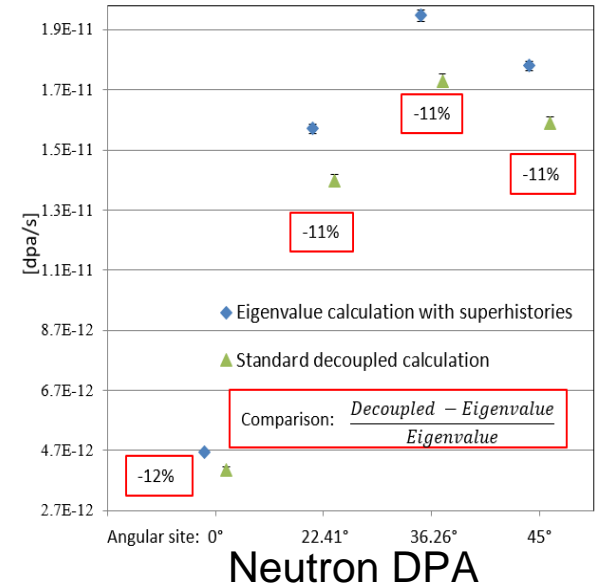
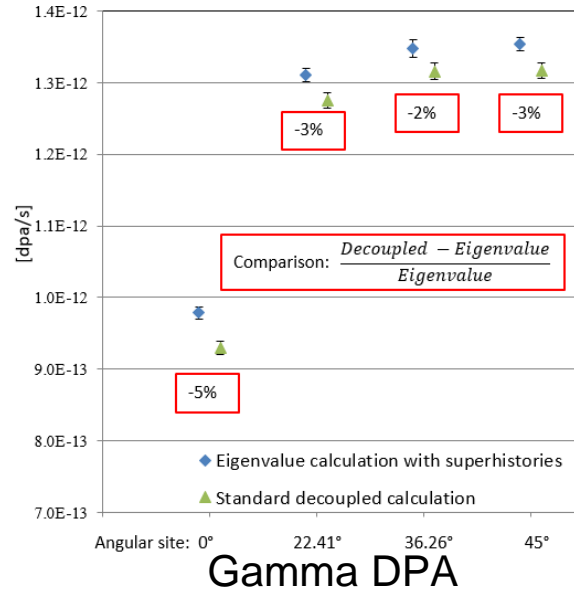
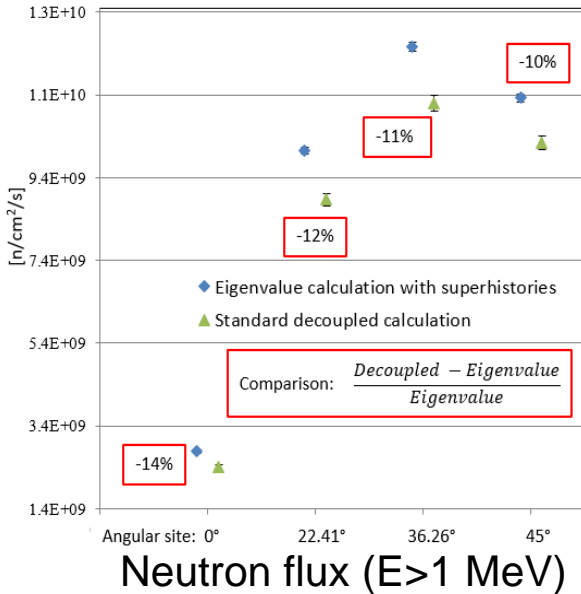
Proposed Approach



Variance Reduction Technique: DSA with multi-response capability

Standard vs. Proposed Approach: Decoupled vs. Single

Reactor system: Gen-III core Objective: n/g dpa damage to vessel



Conclusions and Perspectives

- Aim of the study: approximations introduced in decoupling process
- Interesting underestimation for single eigenvalue simulation
- Impact of fixed source spectrum for decoupled methodology
- Variance Reduction: Direct Statistical Approach (DSA) also for k-simulations

Thanks for your attention