

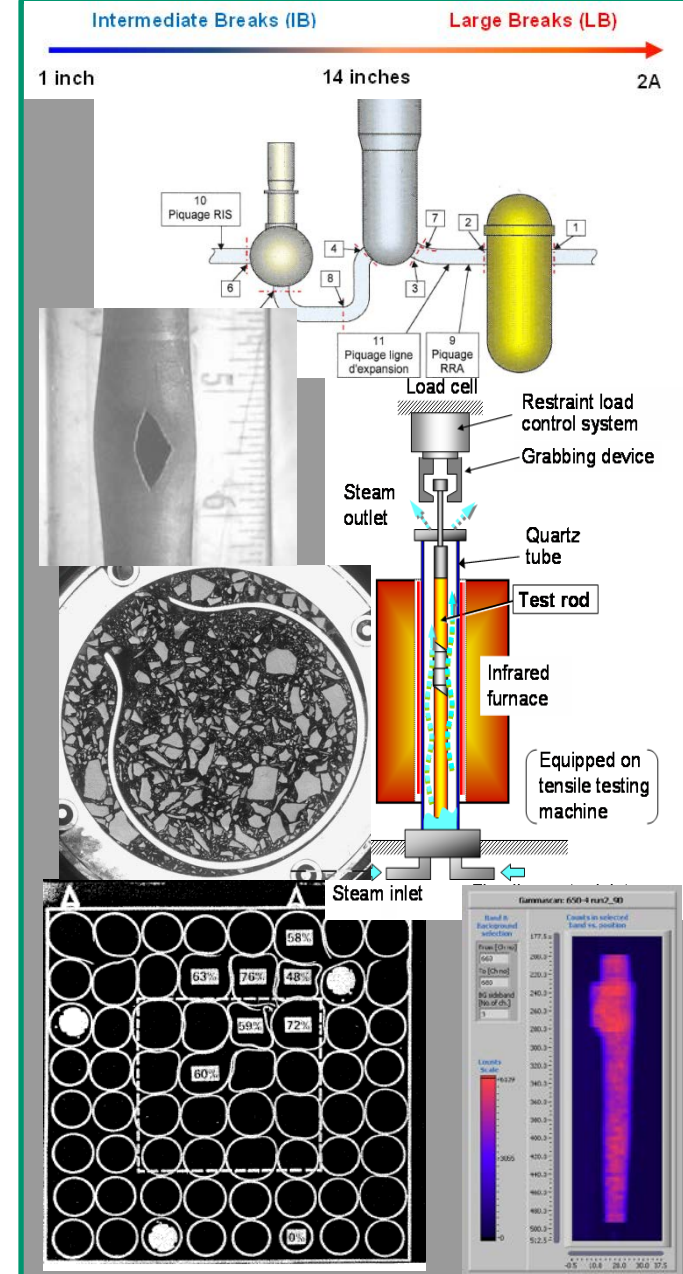
*S. BOUTIN – S. GRAFF – A. BUIRON*

# A New Method Taking into Account Physical Phenomena Related to Fuel Behaviour During LOCA

Seminar 1a - Nuclear Installation Safety - Assessment

# AGENDA

1. Context
2. Development of LOCA reference transients
3. Development of LOCA analysis method
4. How to model fuel behaviour during LOCA
5. Pending questions



# 1 - Context

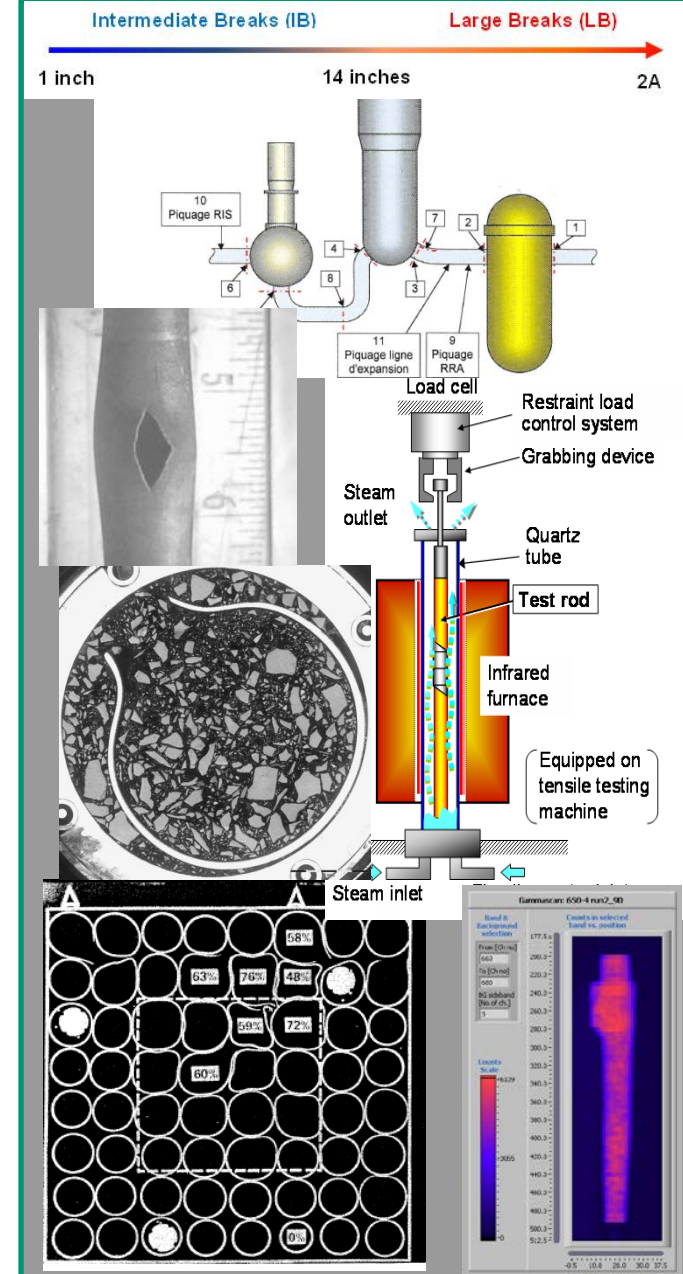
- In **1974, AEC** (now U.S. NRC) promulgated the 10CFR50.46 and its appendix K that define **LOCA reference transients** with regard to the maximal break size, **safety criteria** and **physical phenomena** that have to be considered to demonstrate **core coolability** during LOCA
  - U.S. requirements adopted in France **at the start of the French PWR nuclear program**
  - **Numerous research programs** have addressed the fuel behaviour during LOCA and **new cladding materials** have been introduced in French reactors
- ➡ Because of these evolutions, **the French Nuclear Safety Authority (ASN) has decided to review the LOCA safety demonstration regarding core coolability**

# 1 - Context

- The **Advisory Committee for Reactors** got together during two meetings in **2010** and **2014** in order to
  - Review the **IRSN evaluation of EDF proposals** on the LOCA safety demonstration regarding core coolability
    - (1) **Definition of the LOCA reference transients**
    - (2) **Physical phenomena to be taken into account and LOCA safety requirements associated with safety limits to be verified**
    - (3) **LOCA analysis method assumptions**
  - Issue **recommendations** to the **ASN**
- In 2015/2016, **the new LOCA analysis method**, the so-called **CathSBI**, proposed by EDF was reviewed by IRSN
  - ➔ **This new method will be first applied in 2017** (for the fourth 10-yearly safety review of EDF's 900 MW<sub>e</sub> nuclear reactors, then for the next 10-yearly safety review of EDF's 1300 MW<sub>e</sub> and 1450 MW<sub>e</sub> nuclear reactors)

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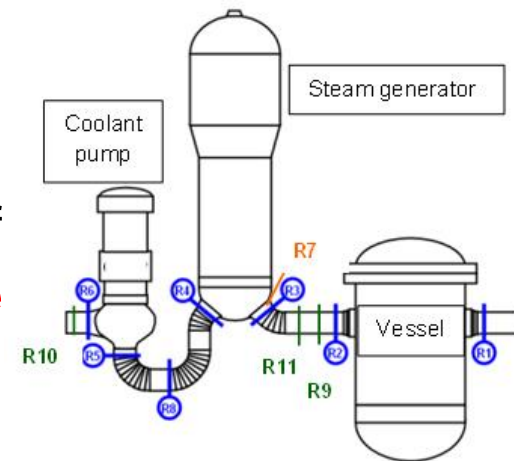
## 2 - Development of LOCA Reference Transients

- **In the current French LOCA safety demonstration**

- The **thermal-hydraulic analysis**, aimed at checking core coolability, is carried out for **breaks of any location** and **any size**: from intermediate to large breaks up to the double-ended guillotine break (called 2A break)



- In contrary, the **mechanical analysis**, aiming at checking the resistance of the internal structures of the reactor vessel and the fuel assemblies, is performed for a selection of **eleven conventional breaks which are mostly IB** (notably 6 guillotine breaks limited in size because of the existence of pipe whip restraints)

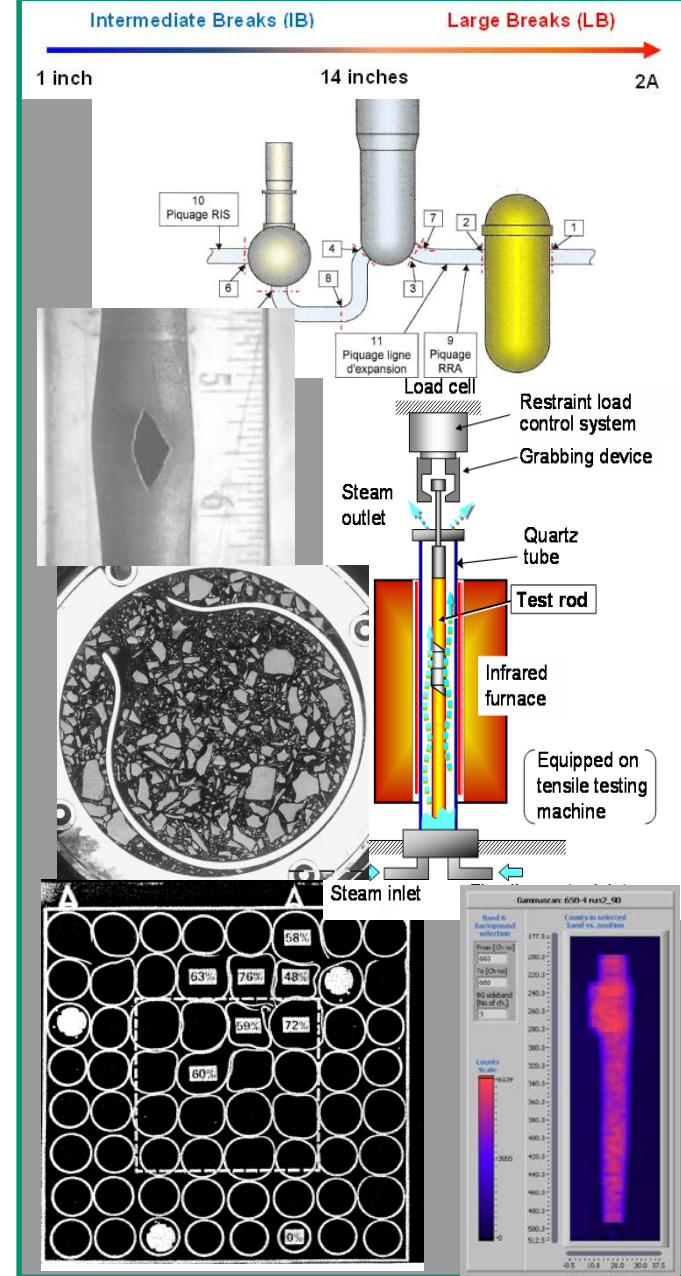


## 2 - Development of LOCA Reference Transients

- **For the newly defined safety LOCA demonstration**
  - **Same break sizes** for both thermal-hydraulic and mechanical analysis
    - Maximum break size < 28 inches - depends on plants type
    - Break opening times obtained by dynamic calculations (instead of the historical 1 ms)
  - **Thermal-hydraulic analysis:** Focus on an **improved modelling of the physical phenomena under IB LOCA conditions** rather than focus on the 2A break
  - **Mechanical analysis:** EDF will have to take into account the **stretch operating conditions effects** for the future LOCA studies

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### 3 - Development of LOCA Analysis Method

- **Current French LOCA analysis method (so-called Deterministic Realistic Methodology)** based on
  - **CATHARE** code (best-estimate thermal-hydraulic system code) associated with **mono-dimensional modelling of the vessel and fuel behaviour**
  - With conservative assumptions for initial and boundary conditions, taking into account **uncertainties** with a **deterministic approach**
- **EDF's new CathSBI method** proposed for **IB LOCA studies** is based on
  - **CATHARE** code with **multidimensional thermal-hydraulics modelling of the vessel** and **enhancement of fuel behaviour modelling** taking into account clad ballooning and burst, flow blockage, contacts between neighbouring rods and fuel relocation
  - A **new statistical approach**

### 3 - Evolution of LOCA Analysis Method

- **New statistical approach**

- Takes into account **elementary uncertainties** affecting the **key parameters** in the calculation of the interest parameters during IB LOCA (i.e. peak cladding temperature)
- Takes into account **coupled effects between key parameters** due to uncertainties propagation
- **Ensures the conservatism** of the safety LOCA studies: in 2014, ASN requested EDF to use a **deterministic way** or an approach ensuring a penalizing range of variation for **the most influential uncertainties** (for example rod internal pressure)

- IRSN analysis focussed on the statistical approach validity -

#### **Main IRSN issues**

- **Some elementary uncertainties** still need **to be justified**
- **Consideration of ASN's request by EDF is not sufficient:** most influential parameters and penalizing approach must be justified

➔ **New statistical approach still needs more robustness**

Key parameters

*Initial and boundary conditions*

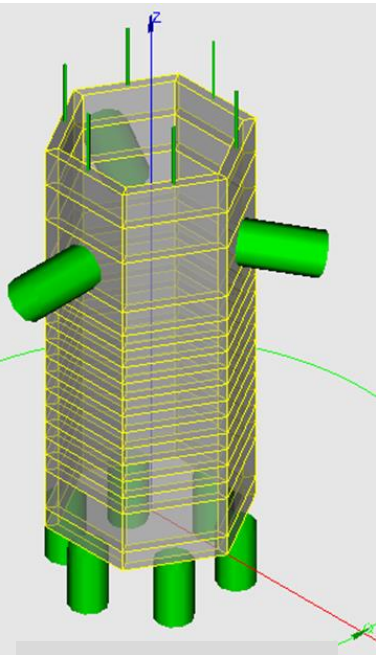
*Code models*

*Variable parameters defining scenario*

### 3 - Development of LOCA Analysis Method

- **Vessel new modelling**

- Use of the CATHARE 3D module is needed in the core and the downcomer to **simulate thermal-hydraulics 3D phenomena**



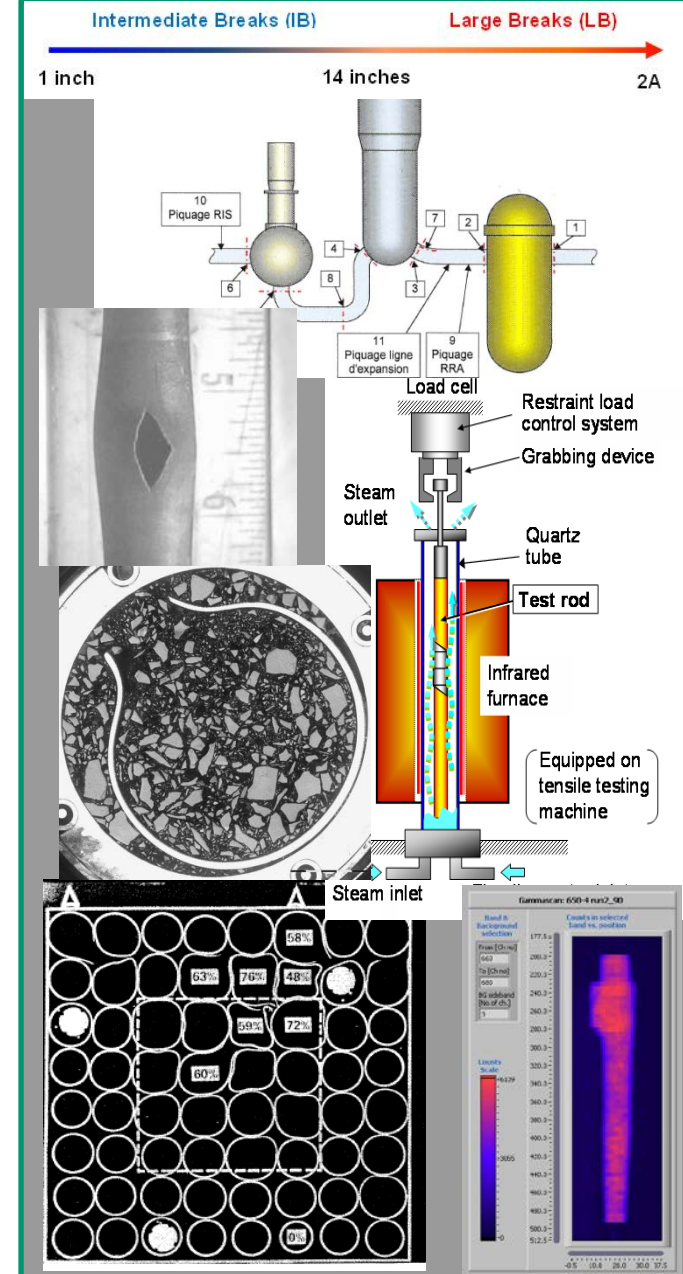
*Downcomer  
model*

- Allow the **cold water injected by ECCS to go down in the boiling downcomer water and to reach the core** (with 1D module, cold ECCS water is “floating” above high void fraction mixture and is lost at the break)
- Model the cross-flows in the gas phase during high-pressure core uncovering → **“Chimney effect”** has a direct positive influence on hot rod cooling
- IRSN analysis focused on CATHARE modelling qualification - **Main IRSN issues**
  - **Significant benefit effect** of vessel new modelling
  - **Lack of validation** of CATHARE 3D module

⇒ **Justifications of vessel modelling choices are still expected and experimental programs are ongoing to validate CATHARE 3D module**

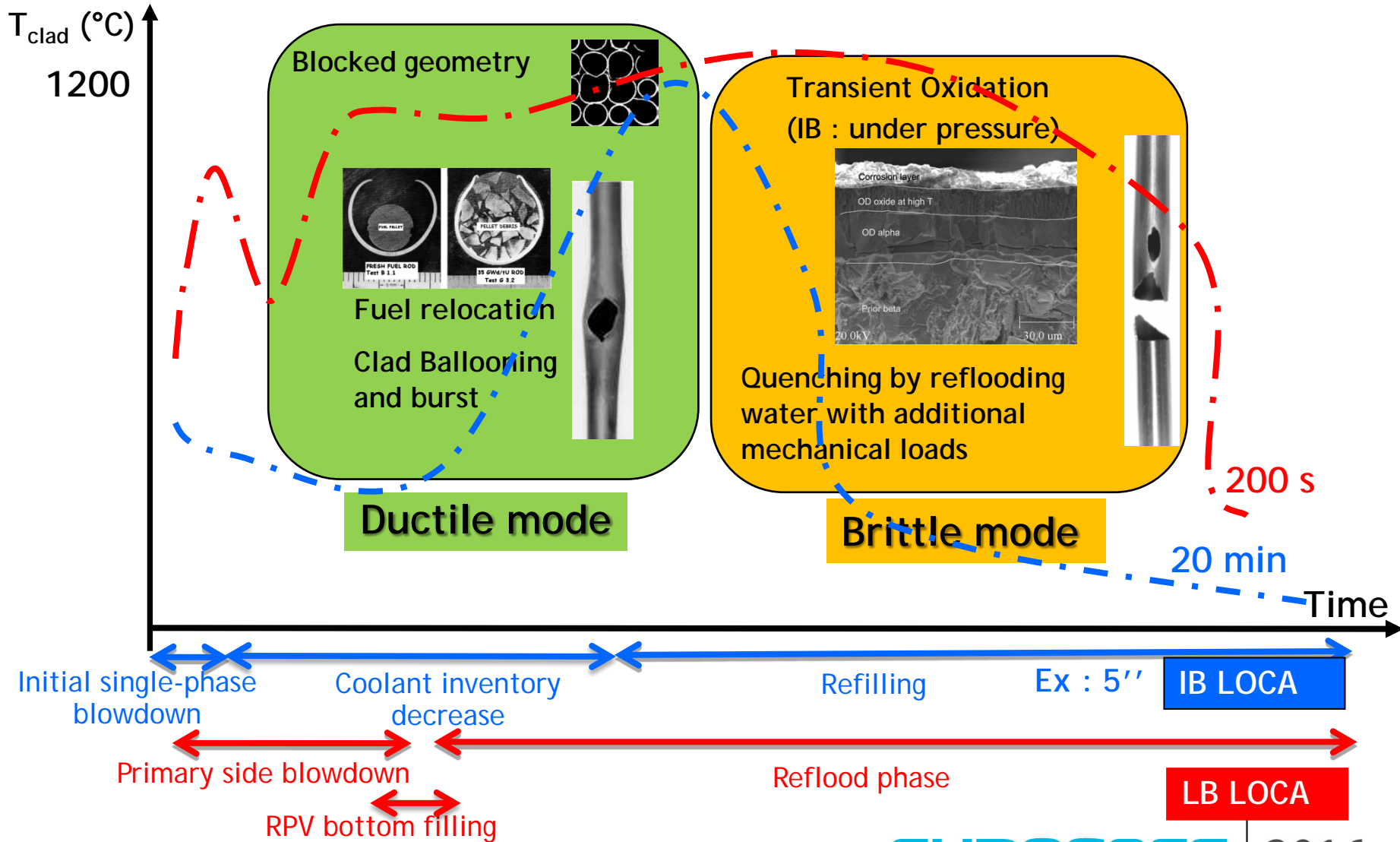
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# 4 - How to Model Fuel Behaviour During LOCA

## ● Phenomenology under IB and LB LOCA



# 4 - How to Model Fuel Behaviour During LOCA

- **New French regulation**

- **EDF calculation approach** before 2014 based on the CATHARE software associated with **1D fuel behaviour modelling** is not able to take into account **contacts between neighbouring rods** and **relocation of irradiated fuel in clad balloons**

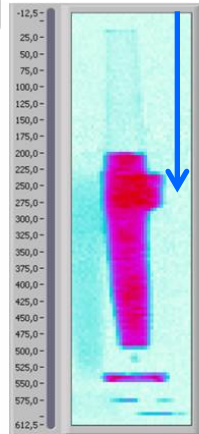
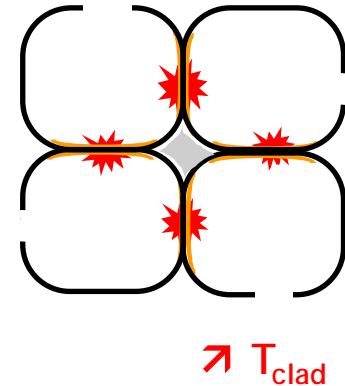
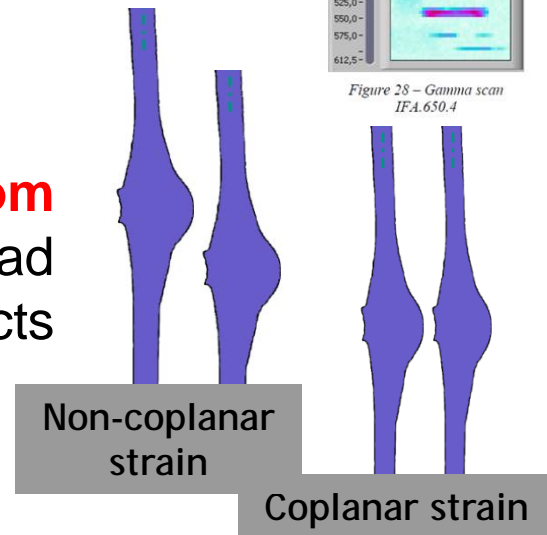


Figure 28 – Gamma scan IFA.650.4

➔ **Cladding temperature is thus underestimated...**

- **CathSBI new method** takes into account

- **Negative effects on heat transfer from cladding to primary coolant** of clad ballooning and burst, blockage and contacts between rods
- Impact of **fuel relocation**



DUCTILE MODE

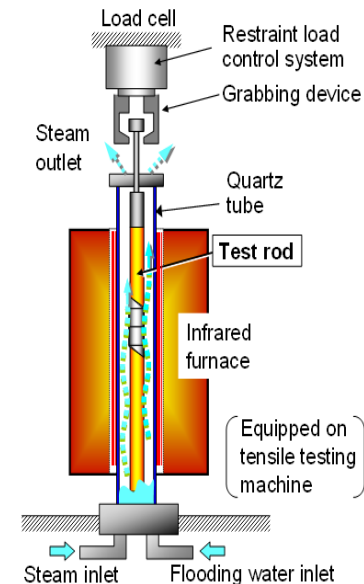
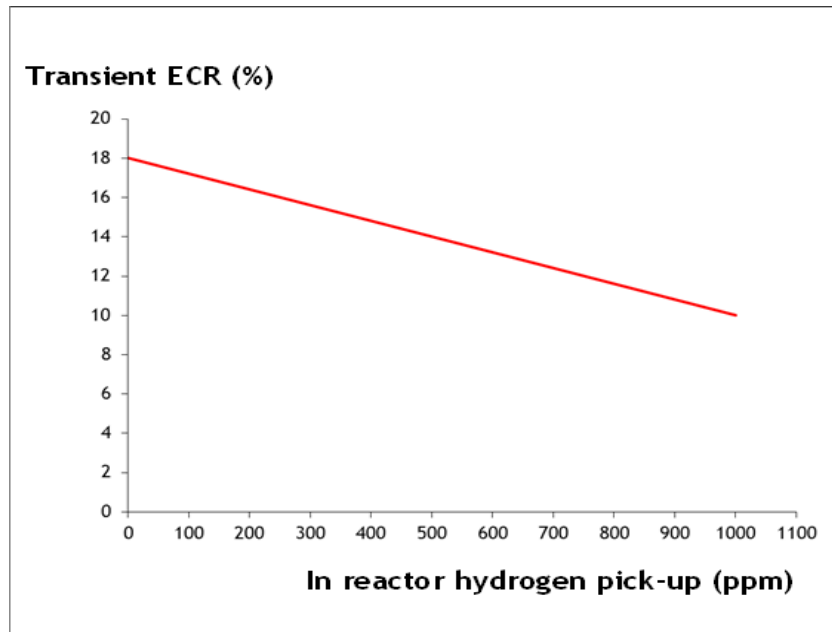


# 4 - How to Model Fuel Behaviour During LOCA

- **New French regulation**

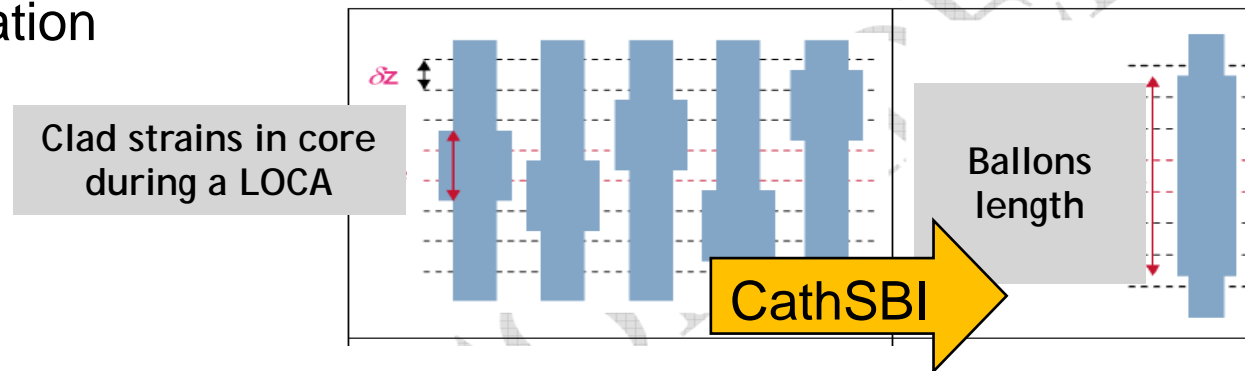
- New safety requirement in 2010 based on **strength-based approach** including an **additional axial loading** during the **quench**
- A new French **ECR criterion**, expressed as a function of **in-reactor hydrogen pick-up** and combined with the **historical 1204°C**, was accepted by ASN in 2014

BRITTLE MODE



## 4 - How to Model Fuel Behaviour During LOCA

- Fuel new modelling is **improved** and **more accurate** by taking into account more physical phenomena that were up to now either mis-modelled or non-modelled
  - **Clad ballooning and burst**: Modelling is underway by EDF to improve the rupture criterion and to cover the IB heating rates
  - **Blockage of fuel channels hydraulics**: Enhancement of the new method consists in taking into account the **balloons length**
    - Subjected to a **statistical approach** based on a range of variation



- **Thermal exchanges** between fuel rods and primary coolant taking into account **the reduction of the exchange surface** due to the contacts between the rods

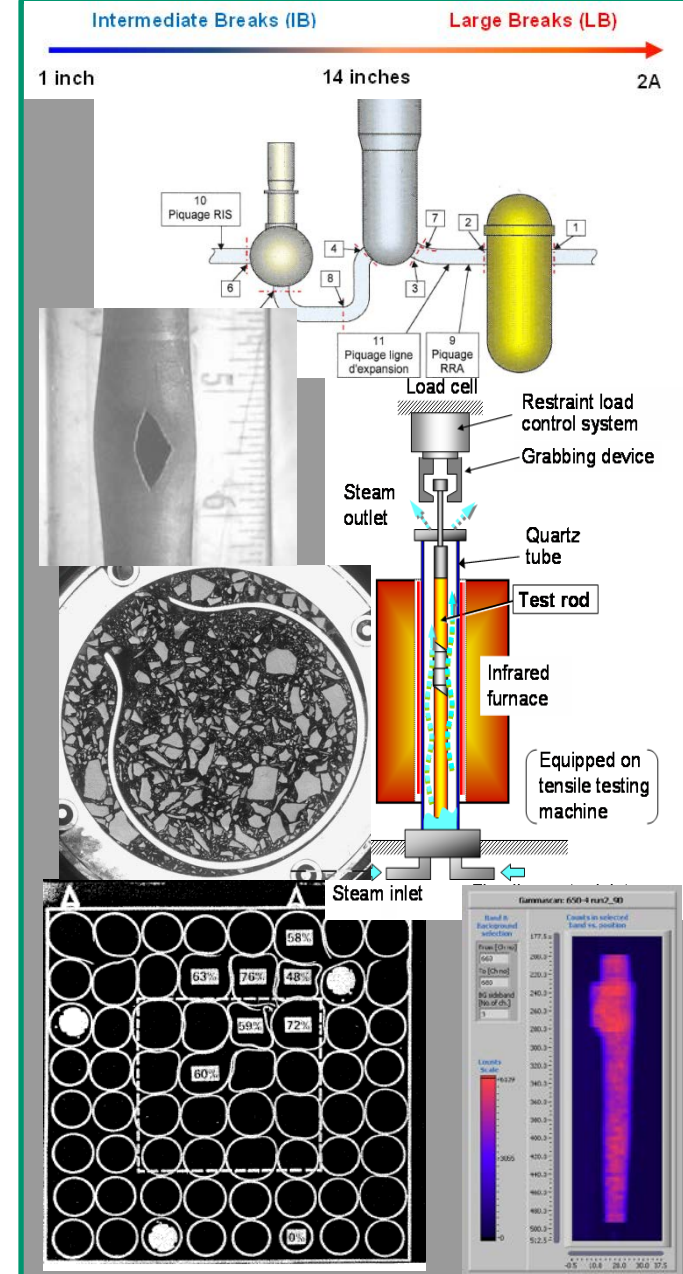


## 4 - How to Model Fuel Behaviour During LOCA

- Fuel new modelling is **improved** and **more accurate** by taking into account more physical phenomena that were up to now either mis-modelled or non-modelled
  - **Fuel relocation phenomenon** (possible accumulation of fuel fragments in the ballooned section of the fuel rods) - various models developed to take into account **relocation consequences on clad temperature**
    - Thermal conductivity of pellet fragments
    - Gap between the pellet fragments and the clad
    - Linear power of the relocated fuel
  - Some parameters of these models are subjected to a **statistical approach based on a range of variation**
- IRSN analysis focused on the statistical approach and fuel modelling qualification - **main IRSN issues**
  - **Lack** of justification for **some uncertainties**

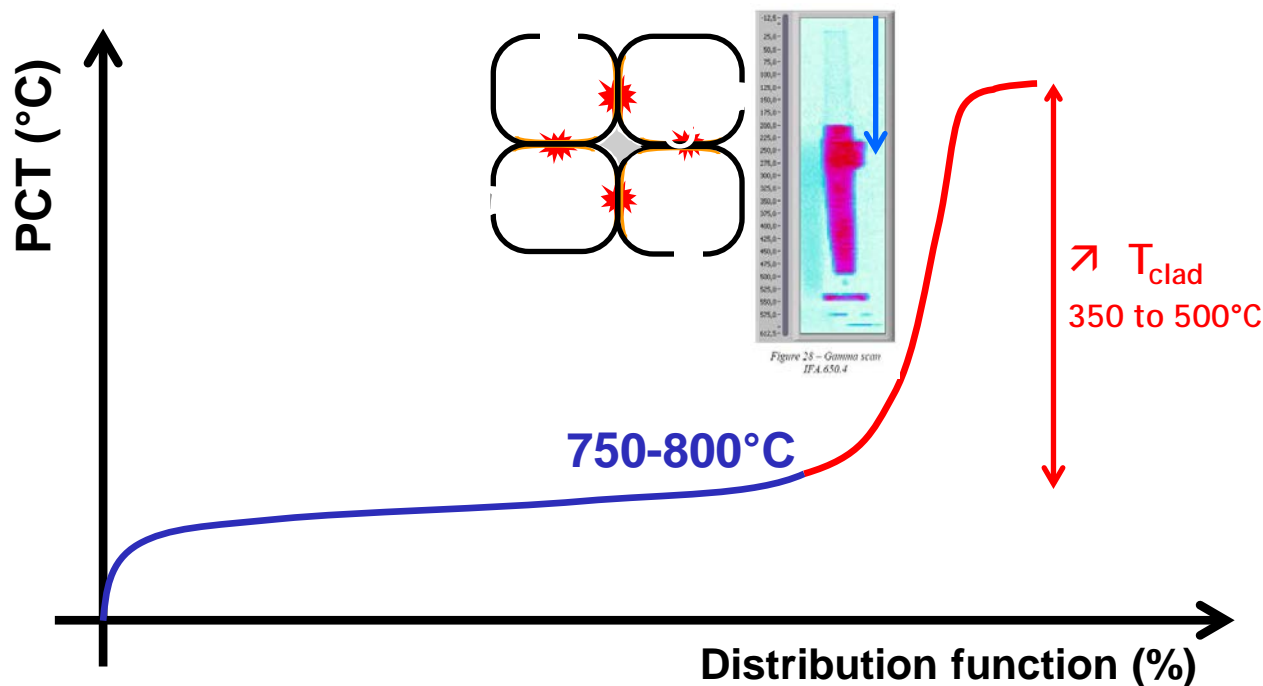
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## 5 - Pending Questions

- PCT and ECR calculated without taking into account all the fuel physical phenomena may be **underestimated**: the first results obtained with CathSBI show a **significant sensitivity** of the **clad temperature** to the input parameters



High sensitivity linked to the fuel rod phenomena activation if

*difference between clad internal and external pressures high enough + clad temperature above 750 – 800 °C*

- ➔ Ensuring the robustness of LOCA safety studies is still a challenge for EDF

Thank you for your attention

