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# FAILURE ASSESSMENT METHODOLOGIES FOR COMPONENTS UNDER SEVERE ACCIDENT LOADING

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### OUTLINE

- Introduction
- Structure mechanics analysis methods for integrity assessment of a PWR
  - coolant loop under a core melt scenario
  - steel containment under peakwise loads (hydrogen combustion)
- Summary and conclusions

#### INTRODUCTION

#### Severe accident scenarios with molten core material





#### Three Mile Island Nuclear Generating Station (TMI)

March 28, 1979

Fukushima Daiichi Nuclear Power Plants

March 11, 2011

### **INTRODUCTION**

- Safety relevance of the integrity of components under severe accident loading
  - primary circuit components
  - containment structures
- Objectives of research work
  - development
  - provision
  - validation

of structural mechanic analysis methods



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#### **SAFETY RELEVANT ISSUE**

#### Primary circuit of German PWR

steam generator

Which component of a primary circuit fails first during a severe accident scenario?









#### Simplified FE-analysis/ASTOR





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#### LOADING CONDITIONS DURING A CORE MELT SCENARIO

Load case "Total Station Blackout" calculated with MELCOR



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Load case "Total Station Blackout" calculated with MELCOR



#### STRUCTURE MECHANICS ANALYSIS MODEL



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#### **MATERIAL PROPERTIES**

#### Temperature dependent stress-strain curves for reactor steel 20 MnMoNi 55 up to uniform elongation



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#### **MATERIAL PROPERTIES**

Temperature and stress dependent creep curves for reactor steel 20 MnMoNi 55 – linear approximation up to 60 % of uniaxial creep failure strain measured by MPA University Stuttgart



### FAILURE CRITERIA FOR INTEGRITY ASSESSMENT

- Failure due to plastification: Uniaxial Uniform Elongation / Stress triaxiality factor TF
- Failure due to creep: Uniaxial failure strain / Stress triaxiality factor TF

$$TF = \frac{\left|\sigma_{1} + \sigma_{2} + \sigma_{3}\right|}{\sigma_{effektiv}}$$
due to Ju and Buttler (1984)

• Safety related assessment of failure:

60% uniaxial creep failure strain, TF >1 based on elasto-plastic stress calculation

 Assessment concerning failure as a matter of fact: 100% uniaxial creep failure strain, TF = 1 for failure due to plastification or/and creep

#### **STRUCTURE MECHANICS ANALYSIS RESULTS**

#### Integrity assessment of main cooling line



#### **STRUCTURE MECHANICS ANALYSIS RESULTS**

Integrity assessment of surge line



#### **STRUCTURE MECHANICS ANALYSIS RESULTS**

Integrity assessment of main cooling and surge line



#### **SAFETY RELEVANT ISSUE**

#### Steel containment of German PWR

What is the load carrying capacity of a steel containment during a severe accident scenario with postulated hydrogen combustion?



## **LOADING DUE TO HYDROGEN COMBUSTION**

- Measured and calculated pressure values in TMI-2 containment during severe accident 1979 [EPRI, 2010]:
  - peak pressure ~0,3 MPa
  - peak duration ~10 s increase / >70 s decrease
- Calculated pressure distributions at top floors in Fukushima units during severe accident 2011 [JNES, 2012]:
  - peak pressure ~1,5 MPa
  - peak duration <100 ms</li>
- Calculated pressure / temperature values for postulated severe accident scenarios with consideration of catalytic recombinators [GRS, 2012]:
  - peak pressures < 0,05 MPa</li>
  - peak duration ~40 70 s
  - peak temperatures < 370 °C</li>

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Loadcase: Pressure peak with Peak pressure 1 MPa Peak duration 32 ms local equivalent stress t = **43,0 ms** 1 500 TIME 0.04300 EFFECTIVE DISP MAG 10.00 STRESS RST CALC 0,8 400 TIME 0.04300 - 350.0 Spannung [ MPa ] Druckstoß [ MPa ] 300.0 43 ms 0,6 300 250.0 - 200.0 Druckstoß - 150.0 - 100.0 - 50.0 Vergl. Spannung 0.4 200 Streckgrenze - 0.0 100 0,2 MAXIMUM **A** 389.1 EG 6, EL 1039, IPT 1 0 0 MINIMUM # 0.2358 EG 2. EL 15432. IPT  $\gamma \rightarrow$ 0 20 40 80 100 60 120 140 160 Zeit [ms]

![](_page_22_Figure_1.jpeg)

Loadcase: Pressure peak with Peak pressure 1 MPa Peak duration 32 ms local equivalent stress t = **58,5 ms** 1 500 TIME 0.05850 EFFECTIVE DISP MAG 10.00 STRESS RST CALC 0,8 400 TIME 0.05850 Spannung [ MPa ] - 350.0 Druckstoß [ MPa ] 300.0 58.5 ms 0,6 300 250.0 - 200.0 Druckstoß - 150.0 - 100.0 - 50.0 Vergl. Spannung 0.4 200 Streckgrenze - 0.0 100 0,2 MAXIMUM A 386.9 EG 6, EL 3251, IPT 1 0 0 MINIMUM X 0.1932 EG 3, EL 28604, IPT  $\gamma \rightarrow$ 0 20 40 60 80 100 120 140 160 Zeit [ ms ]

![](_page_24_Figure_1.jpeg)

Loadcase: Pressure peak with Peak pressure 1 MPa Peak duration 32 ms local equivalent stress t = **75,5 ms** 1 500 TIME 0.07550 EFFECTIVE DISP MAG 10.00 STRESS RST CALC 0,8 400 TIME 0.07550 - 350.0 Spannung [ MPa ] Druckstoß [ MPa ] 300.0 75.5 ms 0,6 300 250.0 - 200.0 Druckstoß - 150.0 - 100.0 - 100.0 - 50.0 - 0.0 Vergl. Spannung 0.4 200 Streckgrenze 100 0,2 MAXIMUM **A** 329.6 EG 6. EL 3458. IPT 1 0 0 MINIMUM # 0.2024 EG 2, EL 38441, IPT  $\gamma \rightarrow$ 0 20 40 60 80 100 120 140 160 Zeit [ms]

![](_page_26_Figure_1.jpeg)

- Loadcase: Pressure peak with
  - Peak pressure 1 MPa

![](_page_27_Picture_3.jpeg)

![](_page_27_Figure_4.jpeg)

Results of parametric study with pressure peak loading

![](_page_28_Figure_2.jpeg)

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#### **DYNAMIC BEHAVIOR OF STEEL CONTAINMENT** Consideration of temperature peak loading

![](_page_29_Figure_1.jpeg)

- Loadcase: Pressure peak with
  - Peak pressure
     1 MPa
  - Peak temperature 1200°C

S

Α

F

Έ

Peak duration
32 ms

Ο

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![](_page_29_Figure_6.jpeg)

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#### **DYNAMIC BEHAVIOR OF STEEL CONTAINMENT** Consideration of temperature peak loading

![](_page_30_Figure_1.jpeg)

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![](_page_31_Figure_0.jpeg)

![](_page_32_Figure_0.jpeg)

![](_page_33_Figure_0.jpeg)

![](_page_34_Figure_0.jpeg)

#### CONCLUSIONS

- Structural behaviour of a PWR cooling loop under loads due to core melt scenarios
  - plastic strains in the main cooling line and the surge line may reach limit values before the RPV heats up
  - structure mechanics results may effect thermal hydraulic results of accident scenarios

code coupling, simplified method in system codes

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#### SUMMARY AND CONCLUSIONS

- Structural behaviour of a PWR cooling loop under loads due to core melt scenarios
  - plastic strains in the main cooling line and the surge line may reach limit values before the RPV heats up
  - structure mechanics results may effect thermal hydraulic results of accident scenarios
    - > code coupling, simplified method in system codes
- Steel containment behaviour under internal peakwise loading
  - oscillations of the pressure loaded area for peak duration 20 50 ms
  - quasi-static behaviour for peaks with duration longer than 100 ms
  - pressure peak values up to 0.4 MPa effect no plastification
  - temperature peaks may effect limited plastification and local failure close to the inner surface